

# ITER In-Vessel Dust and Tritium Control Implementation Status and Plan

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## Outline & Introduction

- **Licensing process of ITER ongoing**
  - Preliminary Safety Analysis Report delivered in March 2010
  - "Décret de Autorisation a la Création de ITER" (~Jan 2012)
  - Three necessary steps: answering to Safety Authority, (2010-11), Public Enquiry (Jun-July 2011), Group Permanent (2011)
  - ITER main safety issues: In-Vessel Dust and Tritium together with ACP in primary cooling water circuits
- **Safety analysis based on in-vessel inventories**
  - **Dust (level of dust accumulation  $\propto$  to the level of erosion)**
    - $\leq 1000$  kg limit of mobilizable dust
    - $\leq 1176$  kg (Be/W) dust on hot surfaces
  - **Tritium (mainly in Be dust and co-deposited layers)**
    - $\leq 1$  kg including tritium in the cryo-pumps

## In-Vessel dust&tritium control strategy

- Measure dust&tritium and remove before reaching the limits
- Maintain the inventory as low as reasonable achievable

### Measurements

- **Dust** : by an overall and local PFC erosion measurements validated progressively via
  - local dust measurement
  - analysis of dust, samples and material recovered from the VV
  - validation of the ratio dust/erosion and of dust production rate model
- **Tritium** : by a tritium tracking procedure assessing periodically the deficit in the inventory (T injected-T recovered-T burnt) supported by
  - Analysis of T in dust, samples and components recovered from the VV
  - Tritium retention measurement of in-vessel component surface
  - Validation of the on-line tritium balance measurements and model

### Removal

- **Dust** : recover dust as much as possible at any shutdown via MPD and also via CTM/CMM during cassette removal
- **Tritium** : PFC and VV baking, other wall conditioning techniques, removal of dust and components from the VV

## Dust & Tritium Measurements

- **overall PFC erosion**
  - **Requirements**
    - 0.5 mm at  $\leq 5$ m distances
    - $\leq 1$ mm at farther distances
  - by IVVS(baseline): R&D and design optimization on-going
  - By other baseline diagnostics (initial studies started)
- **divertor erosion**: laser-based concepts
  - FM LIDAR, Speckle interferometry, Digital holography
  - Studies ongoing, choice before CDR (2013)
- **local dust**: several gauges in divertor cassette area. Two systems under R&D in present tokamaks:
  - Capacitive diaphragm microbalance (ITA-EU, ITPA JEX at KSTAR)
  - Electrostatic grid
- **tritium retention**: laser-induced spectroscopy
  - LIDS, LIBS Studies ongoing, choice before CDR (2012)
- **removable samples**: preliminary design by Feb 2012
  - dust accumulation, erosion/deposition effects and tritium retention

Continuous integration and synergy among Diagnostics, RH, Physics, In-Vessel components and safety necessary

Validation through R&D on going, JET ILW,

ITPA Tasks, IAEA CRP, etc...

and non-nuclear phase of ITER (H2 programme).

## Dust and tritium removal

- **Dust** : Vacuum cleaning
  - via MPD with divertor cassettes in ( $\approx 15\%$  of dust recovered)
  - Via MPD, CTM/CMM at divertor cassette removal ( $\approx 90\%$ )
- **Tritium**
  - Baking of divertor at  $350^\circ\text{C}$ , Blanket at  $240^\circ\text{C}$ , VV at  $200^\circ\text{C}$
  - $\approx 50\%$  of tritium in dust co-deposited layers released
  - Wall cleaning techniques, dust and in-vessel components removal

## ALARA criterion

- All efforts in reducing dust production and tritium retention
- Maintain the radioactive inventories (including uncertainties) as low as possible
  - **Dust**
    - Remove dust as first operation of each shutdown
    - Clean cassettes and components before moving to Hot Cell
  - **Tritium**
    - Baking when necessary and before each VV venting
    - Dedicated plasma operation campaigns if necessary

## Dust on hot PFC surfaces

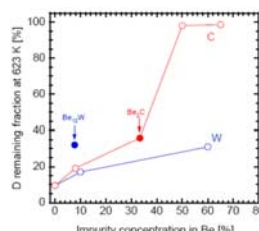
**Safety issue: possible production of H2 in case of in-VV LOCA**  
**Three lines under studies/R&D**

- Detect dust on hot surfaces and remove it (Low limits: 11/76 kg, Be/W)
- Inject inert gas on in-VV on O2 detection following a LOCA
- Model validation to show the non-contemporaneity of ingress of air with PFC high temperatures

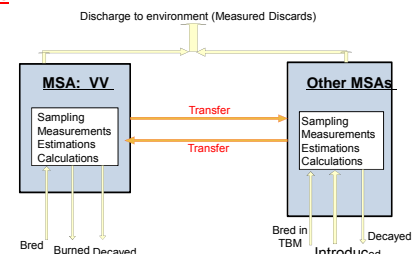
## Examples of efforts on-going



### Tritium retention versus temperature



### Tritium tracking for the Vacuum Vessel



## Few main references

- J. Kim et Al.: Plasma-Wall Interaction Diagnostics in ITER ..., 2011, under review in FS&T
- K. Sugiyama, J.Roth et Al.: Investigation of Deuterium retention and release ....., PSI 2010
- J. Roth et al., J. Nucl. Mater. 390-391, 1 (2009)
- Le Guern et Al.: SOFT 2010
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Acknowledgements: ITER Diagnostic, Physics, RH In-Vessel Components, Tritium and Safety Groups, F4E and EU Associations and PISCES developing most of the relevant R&D.

[www.iter.org](http://www.iter.org)

13th PFMC Workshop / 1st FEMaS Conference, Rosenheim, 8-12 May 2011  
 Presented by Sergio Ciattaglia  
 Title : In-Vessel Dust and Tritium Control, Implementation Status and Plan

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