

Deuterium retention in various pre-damaged tungsten materials as a function of the exposure temperature

B. Tyburska-Püschel^{a,*}, V. Kh. Alimov^b, K. Ertl^a, J. Dörner^a

^aMax-Planck-Institut für Plasmaphysik, EURATOM Association, 85748 Garching, Germany
^bHydrogen Isotope Research Center, University of Toyama, Japan

Motivation

- During Iiter operation, 14 MeV fusion neutrons will not only introduce new traps for tritium in tungsten but also will transmute W into a compound containing about 3% Re after one year.
- To account for these effects, deuterium retention in different batches of damaged tungsten materials exposed to D plasma has been investigated as a function of the exposure temperature.
- Based on these results, the dependence of the D concentration in the radiation-induced defects on the exposure temperature was obtained for W materials and 97W-3Re compound.

Experiment

- All samples were self-irradiated above the saturation dpa (displacements per atom) level for D retention, i.e. 0.25 dpa (for $E_{th} = 90$ eV), using 20 MeV W ions
- The pre-damaged samples were exposed to high-flux, low-energy D plasma at various temperatures ranging from 350 to 750 K.
- The deuterium depth profiles were measured by means of NRA using the $D(^3He,\alpha)p$ reaction. The trap density was determined in the damaged zone.
- The total D retention was measured by TDS.

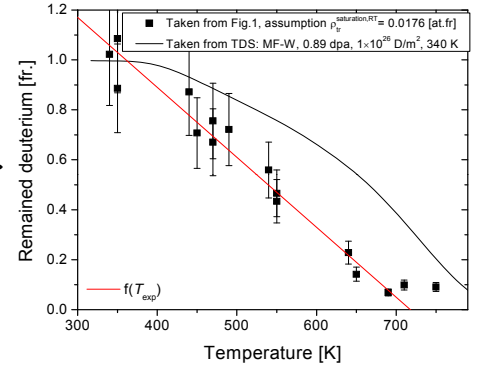
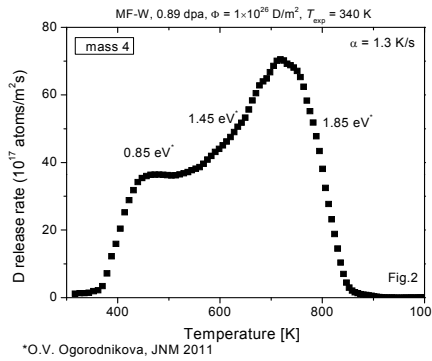
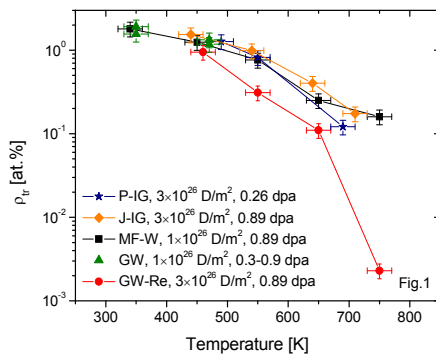
Materials

Plansee Iiter-grade → P-IG	Japanese Iiter-grade W → J-IG	MF-Standard → MF-W	Goodfellow → GW	Goodfellow 3% Re → GW-Re
<ul style="list-style-type: none"> grains perpendicular to the surface grain size 1-10 μm mechanically polished and outgassed at 1200 K for 2 h 	<ul style="list-style-type: none"> grains quasi-perpendicular to the surface grain size 1-5 μm mechanically polished and outgassed at 1470 K for 30 min 	<ul style="list-style-type: none"> grains parallel to the surface grains size 1×(2-5) μm mechanically polished and outgassed at 1200 K for 2 h 	<ul style="list-style-type: none"> grains parallel to the surface structure and grain size similar to MF-W mechanically polished and outgassed at 1200 K for 2 h 	<ul style="list-style-type: none"> grains parallel to the surface grains are much thinner than in MF-W and GW foils 50 μm → no polishing, no thermal treatment

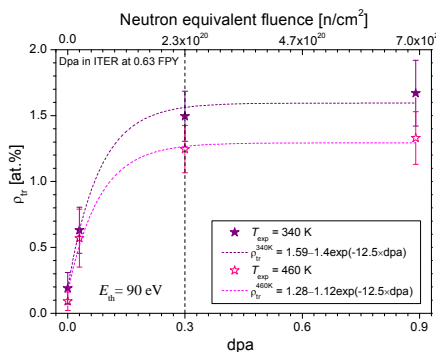
Results

Trap density, ρ_{tr} , dependence on the exposure temperature

Assumptions : all traps are populated, one trap corresponds to one D atom.



ρ_{tr} dependence on the exposure time (dpa)



$$\rho_{tr}(t, T_{exp}) = \rho_{tr}^{saturation, RT} \cdot f(T_{exp}) - 1.4 \cdot f(T_{exp}) \cdot \exp \left[-12.5 \times \frac{N_d \Phi_n^{flux} t}{N_W dpa} \right]$$

$\rho_{tr}^{saturation, RT}$ - ρ_{tr} measured at saturation dpa at RT

$N_d \left[\frac{10^7 displ}{ion \cdot m} \right]$ - no. of displacements

Φ_n^{flux} - neutron flux

N_W - tungsten density

$$f(T_{exp}) = 2.01 - 0.0028 \times T_{exp}$$

- [1] J.C. He et al., JNM 377 (2008) 348–351
- [2] B. Tyburska et al., JNM 395 (2009) 150–155
- [3] A.V. Golubeva et al. JNM 363–365 (2007) 893–897

Summary

- The results indicate that in all materials investigated , D concentration in the radiation-induced defects decreases with increasing exposure temperature.
- In all samples but these with Re, the D concentration is comparable, but for 97W-3Re targets it drops somewhat faster with temperature, a behavior which was not observed in undamaged W containing Re [3].
- The reason for that can be found in TEM observations of neutron-irradiated pure and Re-containing tungsten [1]. These studies show that in the presence of even small amounts of rhenium, the number of defects created during damaging is lower in comparison with pure tungsten.
- Based on these and our previous studies [2], we derived a function for the trap density dependence on the plasma exposure time and wall temperature which can be used for a tritium inventory prediction for Iiter.