



Motivation

retention in microcracks volume) are still actual topics.





Fast droplets are generated at earlier time moments. Smaller velocities are observed for late stage of observation. During intermediate stage both groups of droplets with fast and lower velocities are observed



Heat load to target surfaces vs. incidence of impacting plasma stream. Plasma energy density is 2.4

SEM images of tungsten surface after 5, 20 and 100 QSPA pulses of different heat load T₀ = 200 ^oC



Cellular structure appear in re-solidified tungsten surface. The cell sizes are about 100-300 nm.



SEM views of tungsten surface after 10 PPA plasma pulses with heat load above melting threshold

□ *Major cracks*:

▶ distance between cracks: 0.35-0.6 mm and 0.25-0.7 mm for PPA helium and hydrogen plasma, accordingly; 0.5–1 mm for QSPA hydrogen plasma streams.

→ Width cracks: 4-6 µm for helium and 5-10 µm for hydrogen PPA 4-6 µm for QSPA plasma streams plasma:

 Intergranular micro-cracks ODistance between cracks 15 -20 μm; OWidth cracks about 0.5 μm.

Formation of helium-vacancy complexes causes change of lattice spacing

SEM image of the W-target surface, which was partially destroyed by the pulsed plasma-deuteron streams with in the RPI-IBIS facility.



From this measurements erosion depth was estimated to be below 5 um.



$Q = 0.75 \text{ MJ/m}^2, 20 \text{ pulses}$

submicron- and nano-sized cellular structures in the modified surface layer of exposed W targets



Q= 0.45 MJ/m², 100 pulses



A lot of nano-particles are observed on the

Lattice spacing in the stress-free section of rolled tungsten targets versus the number of pulses of helium (1 - 0.4 $MJm^{-2}(PPA)$; 2 - 0.2 $MJm^{-2}(PPA)$) and hydrogen (3 0.4 MJm^{-2} (PPA); $4\ 0.75\ MJm^{-2}$ (QSPA)) plasmas and reference value (5).

and higher level of residual stresses under helium plasma impact.

Width of macro cracks on tungsten surfaces' of different grades v.s. number of plasma pulses of 0.45 MJ/m^2 .



Future increase of irradiation dose led to fast rise of width of cracks for non-deformed (sintered) targets. Crack width achieved 6 µm for this W-grade exposed by 100 plasma pulses. At the same time width of macro cracks is almost 2 times less for deformed tungsten.



Residual stresses in preheated to 200 °C tungsten targets versus the number of plasma pulses of 0.20 MJ/m^2 ; 0.3 MJ/m^2 , 0.45 MJ/m^2

Conclusions

*****Features of tungsten erosion under repetitive plasma heat loads up to 1.1 MJ/m² lasting 0.25 ms, which are relevant to ITER Type I ELMs, has been investigated. Influence of target inclination and neighborhood W and C as divertor components on the material response to the repetitive plasma heat loads was analyzed. **Cracking thresholds and crack patterns (major- and micro-) in tungsten targets as well as residual stresses** after repetitive plasma pulses have been studied for a deformed W material (Plansee AG). Monolithic tungsten which is manufactured according to this technology is considered as ITER-reference grade. The elongated grain orientation was perpendicular to the surface.

***** Initially, the energy threshold for cracking development is found to be ~0,3 MJ/m² for small number of QSPA K-50 pulses (< 20) of 0.25 ms duration and triangular pulse shape. The clear decrease of energy threshold for cracking development was found for grooving number of repetitive plasma pulses. Even if there are no cracks at all for applied low heat load and small irradiation dose, nevertheless there are appeared cracks on tungsten surface after more than 20 plasma pulses of heat load of 0.2 MJ/m².

★ The Ductile-to-Brittle Transition occurs in the temperature range of 200 °C $\leq T_{DBTT} < 300$ °C. For initial target temperature $T_0 > 300$ C no major cracks are formed on the exposed surface.

* Residual stress grows with increase of energy load. Some relaxation of stresses is observed for large number of pulses. Melting of surface layer essentially add to the relaxation of stresses, the residual stress became very small after 20 pulses with melting. For plasma loads below the melting threshold the residual stress is still essential even after 100 pulses in spite of some initial decease with plasma pulses for Q= 0.45 MJ/m². After many pulses stress value is similar for all applied loads (0.2, 0.3 and 0.45 MJ/m²) being on the level of 200-250 MPa.

* The described studies of plasma surface interactions have demonstrated that the RPI-IBIS facility can successfully be used for studying of the WI and WII spectral lines emitted from the eroded target material and for research on the plasma interaction with targets made of tungsten, which is of great importance for nuclear fusion technology.

Comparison of cracks' width on tungsten surfaces of different grades v.s. number of plasma pulses.



- *****The diffraction profiles lightly changes as a hydrogen plasma exposures below result of melting threshold.
- *****The large density of defects on tungsten surface causes a broadening of line after helium plasma irradiation.
- *****Plasma treatments lead to annealing of material and perfection structure.
- **The value of** a_0 **initially grows, but then** decreases with increasing number of plasma pulses with heat load above melting threshold.